

WHAT IS CLAIMED:

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- 50 07 10
- 40 02 15
- 40 02 20
1. A creep resistant zirconium alloy comprising a coarse grained lath alpha microstructure.
 2. The zirconium alloy as claimed in claim 1 wherein the microstructure includes small second phase precipitates.
 3. The zirconium alloy as claimed in claim 1 wherein the small second phase precipitates have a diameter less than $0.15\mu\text{m}$.
 4. The zirconium alloy as claimed in claim 3 wherein the microstructure is partially recrystallized.
 5. The zirconium alloy as claimed in claim 4 wherein the microstructure is less than 50% recrystallized.
 6. The zirconium alloy as claimed in claim 1 wherein the microstructure has an acicular structure which includes a lath spacing within the range from about 0.5 to about $3.0\mu\text{m}$.
 7. The zirconium alloy as claimed in claim 5 wherein the microstructure is an acicular structure and includes a lath spacing within the range from about 0.5 to about $3.0\mu\text{m}$.
 8. A nuclear fuel cladding comprising an annular layer of the creep resistant zirconium alloy as claimed in claim 1.
 9. A nuclear fuel cladding comprising an annular layer of the creep resistant zirconium alloy as claimed in claim 7.
 10. A method of manufacturing a creep resistant zirconium alloy comprising the steps of:

beta heat treating a zirconium alloy to form a first intermediate;
fast quenching the first intermediate to form a second intermediate;

cold working the second intermediate within the range from about 30% to about 40% to form a third intermediate; and

annealing the third intermediate to effect partial recrystallization.

11. The method as claimed in claim 10 wherein the beta heat treating step has a duration within the range from about 1 to about 10 seconds.

12. The method as claimed in claim 11 wherein the fast quenching step is conducted at a cooling rate within the range from about 20 to about 200°C/second.

13. The method as claimed in claim 12 wherein the annealing step is conducted within the temperature range from about 570°C to about 640°C.

14. A fuel cladding for use in a nuclear reactor for cladding nuclear fuel, comprising:

a tube member; and

15 said tube member comprised of a zirconium alloy, said alloy having a coarse grained lath alpha microstructure.

16. The nuclear fuel cladding as claimed in claim 14, wherein said microstructure includes a lath spacing within the range from about 0.5 to 3.0 μ m.

17. A fuel cladding for use in a nuclear reactor for cladding nuclear fuel, comprising:

20 a tube member; and

25 said tubular member having an inner annular layer, which forms an inner diameter of said tubular member,

26 said annular layer comprised of a zirconium alloy, having a coarse grained lath alpha microstructure.

27. A fuel cladding as claimed in claim 16, wherein said microstructure includes a lath spacing in the range of 0.5 to 3.0 μ m.

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